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Rick J. King Director Nuclear Safety Assurance

October 30, 2002

U. S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, DC 20555

Subject:

River Bend Station - Unit 1

Docket No. 50-458 License No. NPF-47

10 CFR 50.46 (a)(3)(ii) Report

File No.

G9.5

RBG-46033 RBF1-02-0173

Ladies and Gentlemen:

In accordance with 10 CFR 50.46 (a)(3)(ii), River Bend Station (RBS) is required to report changes to or errors discovered in acceptable ECCS evaluation models on an annual basis. RBS is obligated to report the nature of the change or error and its estimated effect on calculated Peak Clad Temperature (PCT).

General Electric/Global Nuclear Fuel (GE/GNF) has issued five Notification Letters (NL-2002-01, 02, 03, 04, and 05) to River Bend Station (RBS) on the impact of errors/changes in the SAFER methodology used by GE/GNF to demonstrate compliance with the requirements of 10CFR50.46 during the past year. A summary of these notifications follows:

AD01

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GE/GNF Notification	Nature of change	Estimated PCT Effect	
		Appendix K	Upper Bound
2002-01	An error was identified in the SAFER analysis which resulted in too low a value being calculated for the core spray injection elevation for the lower core spray sparger. This error affects the coolant inventory of the pool in the upper plenum and the amount of steam quenched by the core spray.	+15	0
2002-02	SAFER LOCA analysis did not properly calculate the effect of the steam dryer pressure drop on the initial inventory of water in the vessel.	0	-5
2002-03	An error in the initialization of the fuel rod gap conductance	0	0
2002-04	A change in the evaluation related to migration of the computer software from a VAX computer to a DEC ALPHA computer.	0	0
2002-05	An error in the calculation of the free volume of the downcomer.	0	0
Sum of absolute magnitude of errors during this period		15	5
Cumulative sum of absolute magnitude of errors prior to this period		5	5
Cumulative sum of absolute magnitude of errors including this period		20	10

The Cycle 11 core at RBS consists of mixed GE11 and ATRIUM-10 fuel bundles. Only the PCT associated with GE11 fuel is affected by these notices. The PCT of the ATRIUM-10 is not affected. The LOCA analysis results continue to meet the acceptance criteria of 10CFR50.46(b)(1) that the PCT shall not exceed 2200 deg F. In addition no reanalysis is required per 10CFR50.46(a)(3)(ii).

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In summary, the LOCA analysis results for PCT continue to meet the acceptance criteria and there is no safety concern.

For further information, contact Mr. B. M. Burmeister at (225) 381-4148.

Sincerely,

cc:

U. S. Nuclear Regulatory Commission

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